

Background measurements at Hartlepool Nuclear Power Station to assess the viability of on-site-antineutrino detection.

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1. Introduction & Background

- **Nuclear reactors** used as source of antineutrinos in numerous experiments and proposed for safeguards applications: spectral evolution can estimate reactor fuel burnup and time-averaged power.
- **Antineutrinos** arise from beta-decays of fission product daughters, with an average number of decays (and thus antineutrinos) of about three per daughter and six per fission.
- **A potential site** for placement of an antineutrino detector is at **Hartlepool Nuclear Power Station (NPP)** in the UK.
- At Hartlepool antineutrino signal governed by: position, burnup, power, and various **backgrounds**. The latter are difficult to separate by the reactor signal.
- **The goal of this work:** measure gamma and neutron backgrounds at Hartlepool, to help separate future reactor antineutrino signal from accidental background.

2. Measurements at Hartlepool Advanced Gas cooled NPP

- **Hartlepool NPP (Figure 1)** has two active cores with:

- ~1575 MW (thermal).
- Graphite moderator
- Carbon dioxide cooler
- Uranium dioxide fuel



- **5 gamma and neutron background measurements: 4 around the site and 1 off-site.**

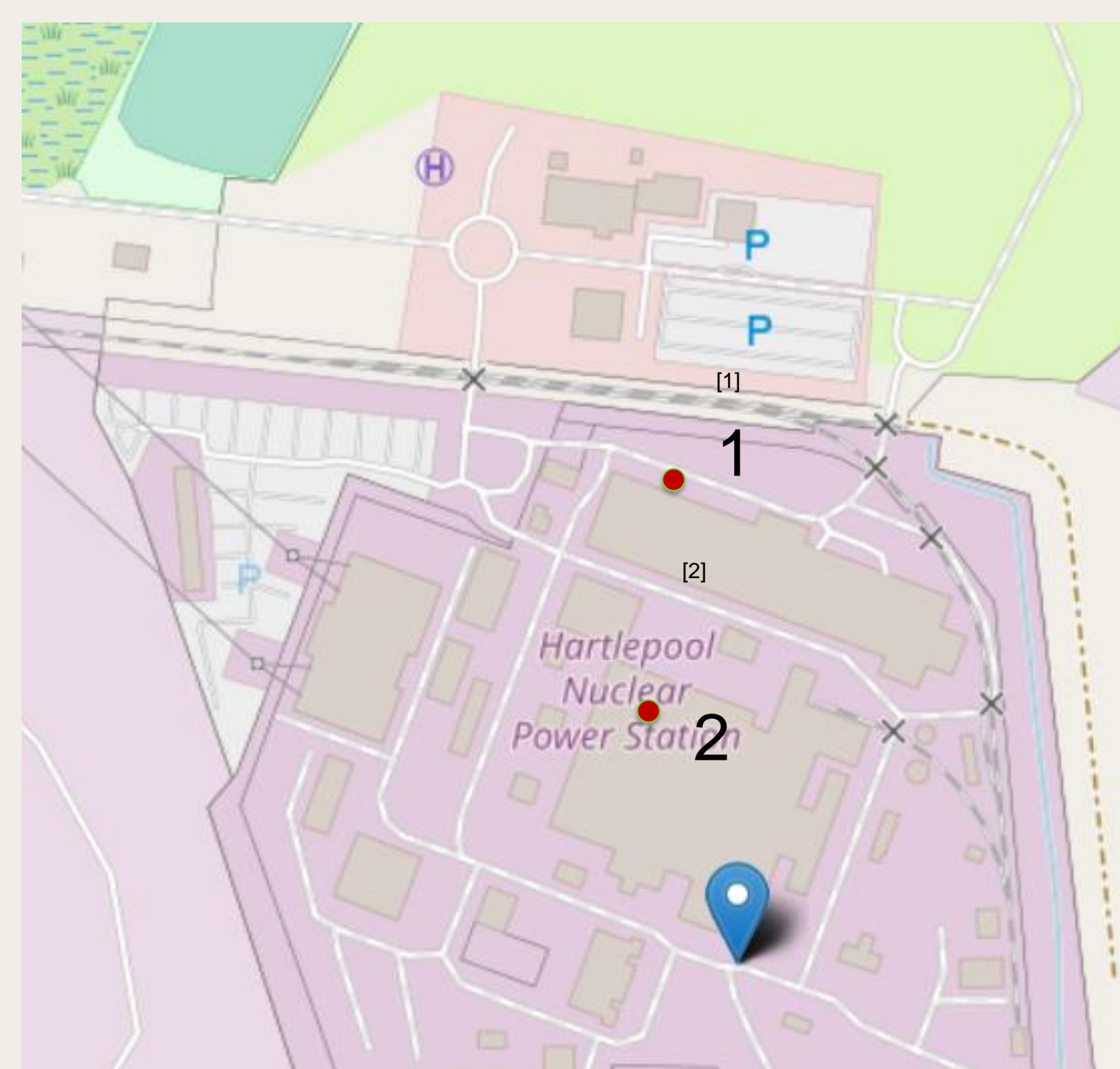


Figure 2: Map of Hartlepool Nuclear Power Station with red dots marking the buildings where measurements were taken.

- [1] Administration Building – 1 measurement performed here
- [2] Reactor Building – 3 measurements performed here

3. Measurements Methodology

Two detectors used for backgrounds:

- for gamma: ORTEC trans-SPEC-DX-100T HpGe
- for neutron: Kromek TN-15 scintillation detector with a state-of-the-art Silicon photomultiplier.

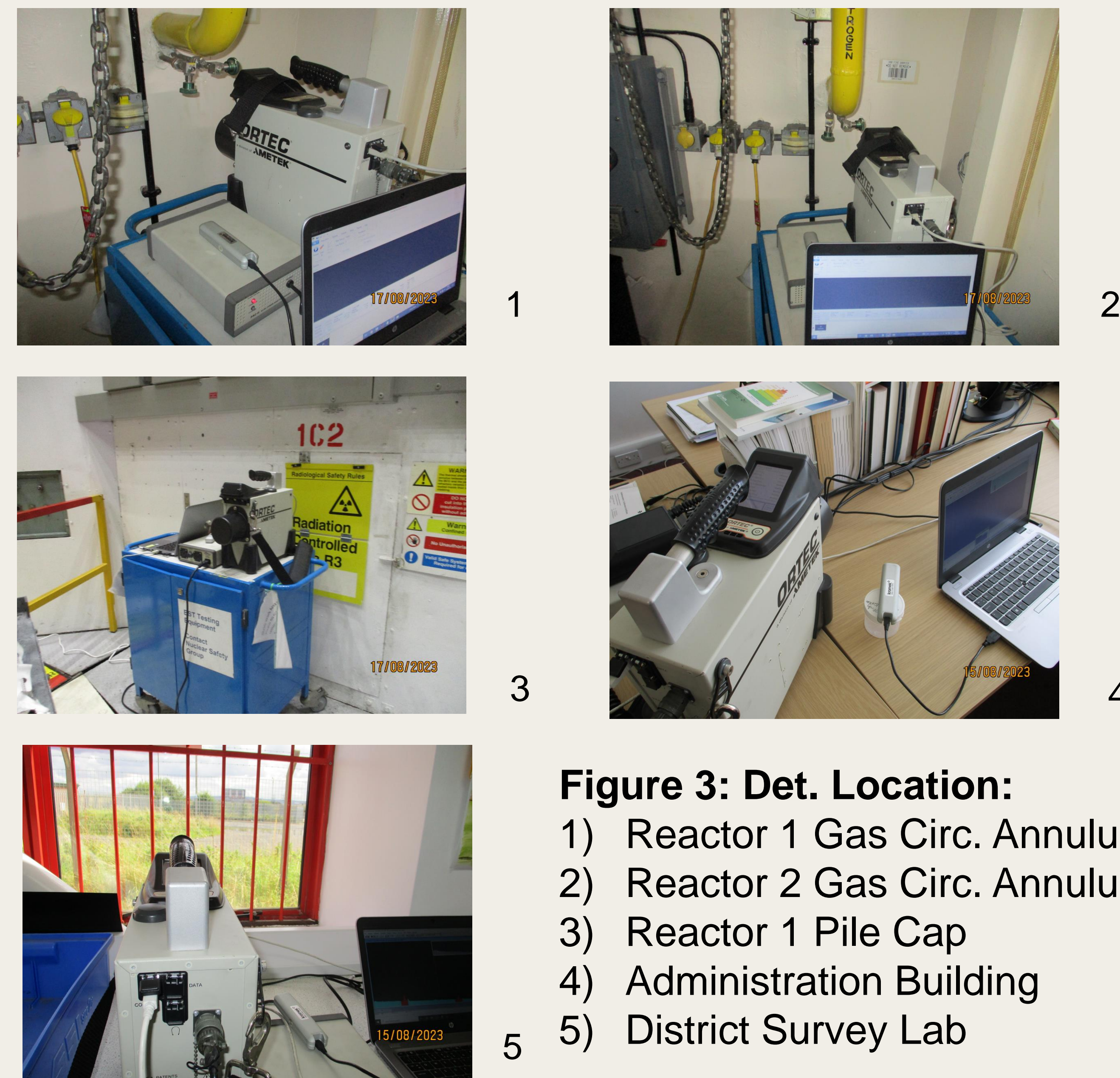


Figure 3: Det. Location:

- 1) Reactor 1 Gas Circ. Annulus
- 2) Reactor 2 Gas Circ. Annulus
- 3) Reactor 1 Pile Cap
- 4) Administration Building
- 5) District Survey Lab

Gamma spectrum calculated by:

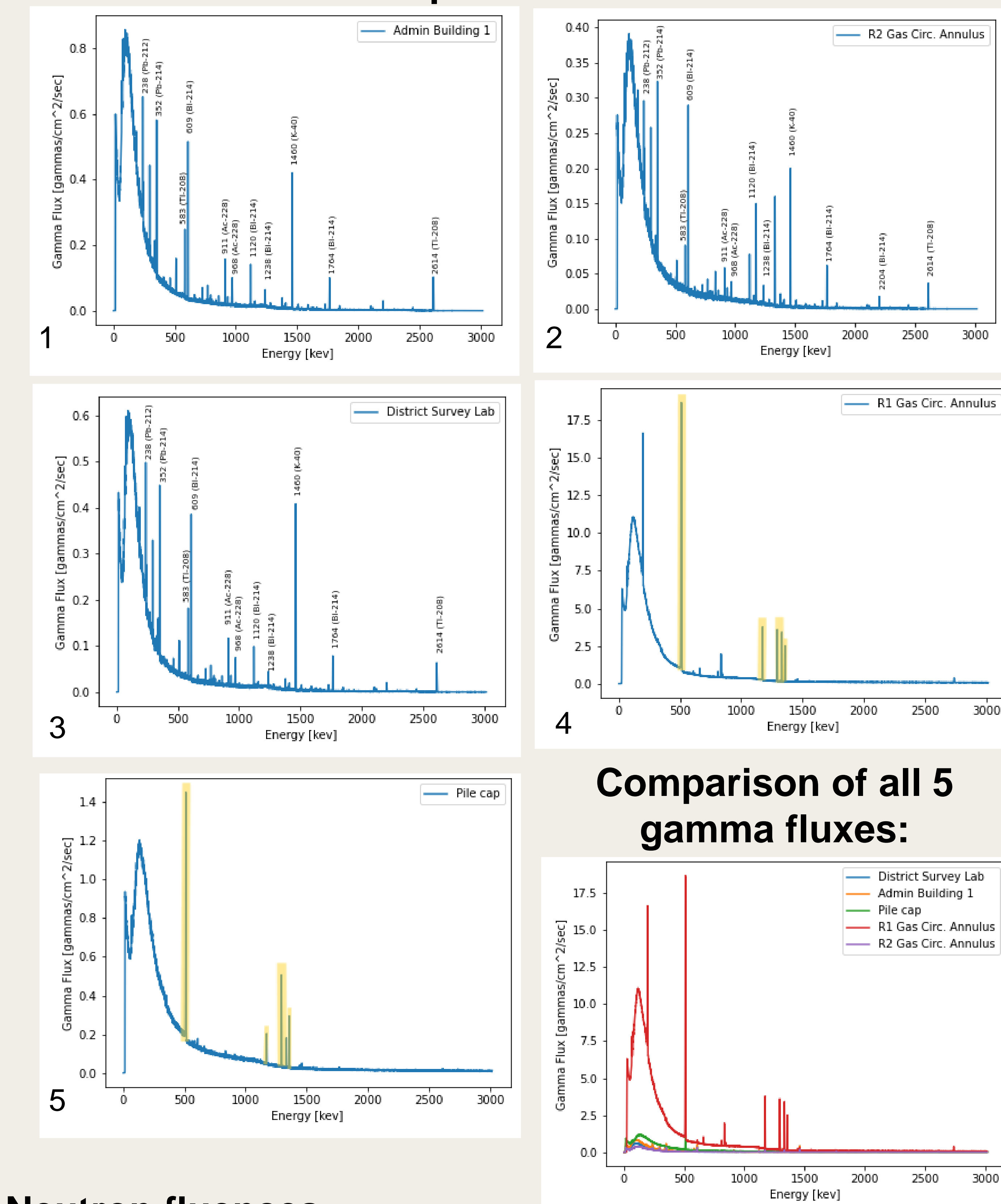
- Plotting calibration curve to find energy level for each channel.
- Dividing counts by time of each measurement acquisition to get count rate.
- Dividing count rate by intrinsic detector efficiency to get gamma flux in units of gammas/(cm²/sec).

Integrated thermal neutron fluence:

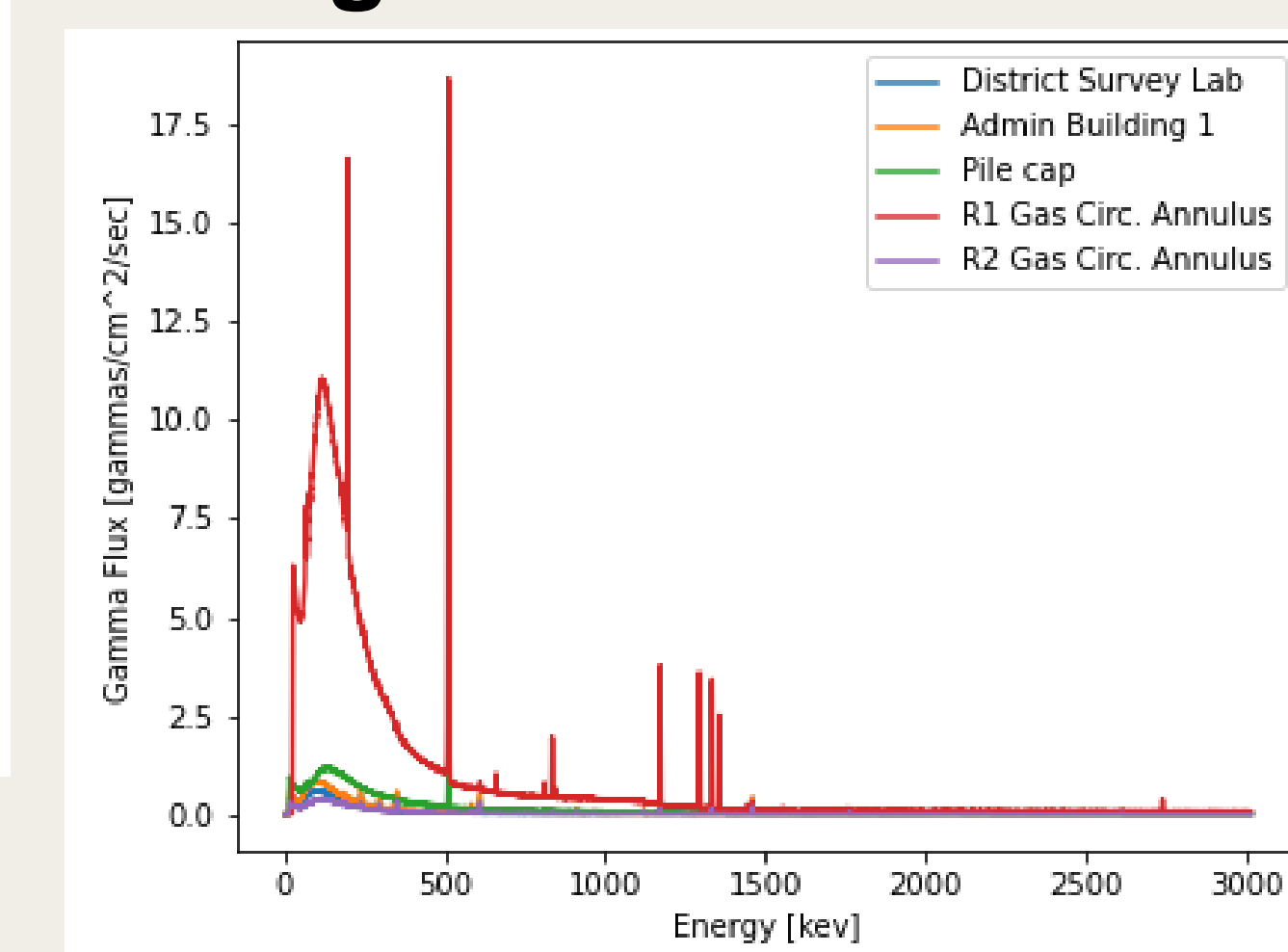
- Dividing total number of counts by the time of acquisition to find count rate.
- Dividing the neutron count rate by sensitivity of the detector and multiplied by the weight of the detector to calculate the neutron flux in units of neutrons/(cm²/sec).

4. Results and Conclusions

Gamma Spec in the 5 location:



Comparison of all 5 gamma fluxes:



Neutron fluences :

- 1) 0.00019 neutrons/cm²/sec
- 2) 0.0002 neutrons/cm²/sec
- 3) 0.00456 neutrons/cm²/sec
- 4) 0.00075 neutrons/cm²/sec
- 5) 2e-05 neutrons/cm²/sec

5. Future Work

More measurements will be performed for: **muon** and **fast neutron** at the same locations to further help identify the total accidental background for future antineutrino signals.